

September 2, 2008

Mr. Charles G. Pardee
Chief Nuclear Officer and
Senior Vice President
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: CLINTON POWER STATION FIRE PROTECTION TRIENNIAL BASELINE
INSPECTION NRC INSPECTION REPORT 05000461/2008006(DRS)

Dear Mr. Pardee:

On July 25, 2008, the U. S. Nuclear Regulatory Commission (NRC) completed an inspection at your Clinton Power Station. The enclosed report documents the inspection results, which were discussed on July 25, 2008, with Mr. M. E. Kanavos and other members of your staff.

The Fire Protection Triennial Baseline Inspection was conducted in accordance with NRC Inspection Procedure 71111.05T, "Fire Protection (Triennial)," dated April 21, 2006. The fire protection inspection team examined activities conducted under your license related to safety and to compliance with the Commission's rules and regulations, and with the conditions of your license related to fire protection and post-fire safe shutdown. The inspection consisted of a selected examination of procedures and records, observations of activities and installed plant systems, and interviews with personnel.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records System (PARS) component of NRC's Agencywide

C. Pardee

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Documents Access and Management System (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Robert C. Daley, Acting Chief
Engineering Branch 3
Division of Reactor Safety

Docket No. 50-461
License No. NPF-62

Enclosure: Inspection Report No. 05000461/2008006(DRS)
w/Attachment: Supplemental Information

cc w/encl: Site Vice President - Clinton Power Station
Plant Manager - Clinton Power Station
Regulatory Assurance Manager - Clinton Power Station
Chief Operating Officer and Senior Vice President
Senior Vice President - Midwest Operations
Senior Vice President - Operations Support
Vice President - Licensing and Regulatory Affairs
Director - Licensing and Regulatory Affairs
Manager Licensing - Clinton, Dresden and Quad Cities
Associate General Counsel
Document Control Desk - Licensing
Assistant Attorney General
J. Klinger, State Liaison Officer,
Illinois Emergency Management Agency
Chairman, Illinois Commerce Commission

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Document Control Desk - Licensing
Assistant Attorney General
J. Klinger, State Liaison Officer,
Illinois Emergency Management Agency
Chairman, Illinois Commerce Commission

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DOCUMENT: G:\DRS\Work in Progress\Clinton 2008-006 DRS BXJ.doc

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Letter to Mr. Charles G. Pardee from Mr. Robert C. Daley dated September 2, 2008.

SUBJECT: CLINTON POWER STATION FIRE PROTECTION TRIENNIAL BASELINE
INSPECTION NRC INSPECTION REPORT 05000461/2008006 (DRS)

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U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No: 50-461

License No: NPF-62

Report No: 05000461/2008006(DRS)

Licensee: Exelon Generation Company, LLC

Facility: Clinton Power Station

Location: Clinton, IL

Dates: July 7 – 11 and July 21 – 25, 2008

Inspectors: Benny Jose, Senior Projects Inspector (Lead)
George Hausman, Senior Reactor Inspector
Robert Winter, Reactor Inspector

Approved by: Robert C. Daley, Acting Chief
Engineering Branch 3
Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

IR 05000461/2008006; 07/07/2008 - 07/25/2008; Clinton Power Station; Fire Protection Triennial Baseline Inspection.

This report covers an announced fire protection triennial baseline inspection. The inspection was conducted by Region III inspectors in accordance with the U. S. Nuclear Regulatory Commission's (NRC's) Inspection Procedure (IP) 71111.05T, "Fire Protection (Triennial)," dated April 21, 2006. Based on the results of this inspection, no findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process." Findings for which the significance determination process does not apply may be Green or be assigned a severity level after NRC management review. The NRC describes its program for overseeing the safe operation of commercial nuclear power reactors in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

A. NRC-Identified and Self-Revealing Findings

Cornerstone: Initiating Events

No findings of significance were identified.

Cornerstone: Mitigating Systems

No findings of significance were identified.

B. Licensee-Identified Violations

No violations of significance were identified.

REPORT DETAILS

1. REACTOR SAFETY

Cornerstones: Initiating Events and Mitigating Systems

1R05 Fire Protection (71111.05T)

The purpose of the fire protection triennial baseline inspection was to conduct a design-based, plant specific, risk-informed, onsite inspection of the licensee's fire protection program's defense-in-depth elements used to mitigate the consequences of a fire. The fire protection program shall extend the concept of defense-in-depth to fire protection in plant areas important to safety by:

- preventing fires from starting;
- rapidly detecting, controlling and extinguishing fires that do occur; and
- providing protection for structures, systems, and components important to safety so that a fire that is not promptly extinguished by fire suppression activities will not prevent the safe shutdown of the reactor plant.

The inspectors' evaluation focused on the design, operational status, and material condition of the reactor plant's fire protection program and post-fire safe shutdown systems. The objectives of the inspection were to assess whether the licensee had implemented a fire protection program that: (1) provided adequate controls for combustibles and ignition sources inside the plant; (2) provided adequate fire detection and suppression capability; (3) maintained passive fire protection features in good material condition; (4) established adequate compensatory measures for out-of-service, degraded or inoperable fire protection equipment, systems or features; (5) ensured that procedures, equipment, fire barriers and systems exist so that the post-fire capability to safely shut down the plant was ensured; (6) included feasible and reliable operator manual actions when appropriate to achieve safe shutdown; and (7) identified fire protection issues at an appropriate threshold and ensured these issues were entered into the licensee's problem identification and resolution program.

In addition, the inspectors' review and assessment focused on the licensee's post-fire safe shutdown systems for selected risk-significant fire areas. Inspector emphasis was placed on determining that the post-fire safe shutdown capability and the fire protection features were maintained free of fire damage to ensure that at least one post-fire safe shutdown success path was available. The inspection was performed in accordance with U. S. Nuclear Regulatory Commission (NRC) Inspection Procedure (IP) 71111.05T, "Fire Protection (Triennial)," dated April 21, 2006. The NRC regulatory oversight process IP used a risk-informed approach for selecting the fire areas and/or fire zones and attributes to be inspected. The inspectors with assistance from a senior reactor analyst used the licensee's Individual Plant Examination for External Events (IPEEE) to select several risk-significant areas for detailed inspection and review. The fire areas and/or fire zones selected for review during this inspection are listed below and constitute five inspection samples.

<u>Fire Area</u>	<u>Fire Zone</u>	<u>Description</u>
A2-n		Division 1 Switchgear Room
CB-4		Division 1 Cable Spreading Room
D-2		Division 1 Emergency Diesel Generator (EDG) Diesel Storage Tank Room
	CB-1i	Control Building EL. 825' General Access Area
	CB-1f	Control Building EL. 762' General Access Area

.1 Shutdown from Outside Main Control Room

a. Inspection Scope

The inspectors reviewed the functional requirements identified by the licensee as necessary for achieving and maintaining hot and cold shutdown conditions to ensure that at least one post-fire safe shutdown success path was available in the event of fire in each of the selected fire areas and for alternative shutdown in the case of control room evacuation. The inspectors reviewed the plant systems required to achieve and maintain post-fire safe shutdown to determine if the licensee had properly identified the components and systems necessary to achieve and maintain safe shutdown conditions for each fire area selected for review. Specifically, the review was performed to determine the adequacy of the systems selected for reactivity control, reactor coolant inventory makeup, reactor heat removal, process monitoring, and support system functions. The review also included the fire safe shutdown analysis to ensure that all required components in the selected systems were included in the licensee's safe shutdown analysis.

The inspectors reviewed the licensee's post-fire safe shutdown analysis, normal and abnormal operating procedures, piping and instrumentation drawings, electrical drawings, their updated final safety analysis report, and other supporting documents to verify that hot and cold shutdown could be achieved and maintained from outside the control room for fires that rely on shutdown from outside the control room. This review included verification that shutdown from outside the control room could be performed both with and without the availability of offsite power.

The inspectors also examined the operators' ability to perform the necessary manual actions for achieving safe shutdown by reviewing post-fire shutdown procedures, the accessibility of safe shutdown equipment, and the available time for performing the actions.

The inspectors reviewed the updated final safety analysis report and the licensee's engineering and/or licensing justifications (e.g., NRC guidance documents, license amendments, technical specifications, safety evaluation reports, exemptions, and deviations) to determine the licensing basis.

b. Findings

No findings of significance were identified.

.2 Protection of Safe Shutdown Capabilities

a. Inspection Scope

For each of the selected fire areas, the inspectors reviewed the fire hazards analysis, safe shutdown analysis, and supporting drawings and documentation to verify that safe shutdown capabilities were properly protected.

The inspectors reviewed the licensee procedures and programs for the control of ignition sources and transient combustibles to assess their effectiveness in preventing fires and in controlling combustible loading within limits established in the fire hazards analysis. The inspectors performed plant walkdowns to verify that protective features were being properly maintained and administrative controls were being implemented.

The inspectors also reviewed the licensee's design control procedures to ensure that the process included appropriate reviews and controls to assess plant changes for any potential adverse impact on the fire protection program and/or post-fire safe shutdown analysis and procedures.

b. Findings

No findings of significance were identified.

.3 Passive Fire Protection

a. Inspection Scope

For the selected fire areas, the inspectors evaluated the adequacy of fire area barriers, penetration seals, fire doors, electrical raceway fire barriers, and fire rated electrical cables. The inspectors observed the material condition and configuration of the installed barriers, seals, doors, and cables. The inspectors reviewed approved construction details and supporting fire tests. In addition, the inspectors reviewed license documentation, such as NRC safety evaluation reports, and deviations from NRC regulations and the National Fire Protection Association codes to verify that fire protection features met license commitments.

The inspectors walked down accessible portions of the selected fire areas to observe material condition and the adequacy of design of fire area boundaries (including walls, fire doors, and fire dampers) to ensure they were appropriate for the fire hazards in the area.

The inspectors reviewed the installation, repair, and qualification records for a sample of penetration seals to ensure the fill material was of the appropriate fire rating and that the installation met the engineering design.

b. Findings

No findings of significance were identified.

.4 Active Fire Protection

a. Inspection Scope

For the selected fire areas, the inspectors evaluated the adequacy of fire suppression and detection systems. The inspectors observed the material condition and configuration of the installed fire detection and suppression systems. The inspectors reviewed design documents and supporting calculations. In addition, the inspectors reviewed license basis documentation, such as, NRC safety evaluation reports, deviations from NRC regulations, and the National Fire Protection Association codes to verify that fire suppression and detection systems met license commitments.

b. Findings

No findings of significance were identified.

.5 Protection from Damage from Fire Suppression Activities

a. Inspection Scope

For the selected fire areas, the inspectors verified that redundant trains of systems required for hot shutdown would not be subject to damage from fire suppression activities or from the rupture or inadvertent operation of fire suppression systems including the effects of flooding. The inspectors conducted walkdowns of each of the selected fire areas to assess conditions, such as, the adequacy and condition of floor drains, equipment elevations, and spray protection.

b. Findings

No findings of significance were identified.

.6 Alternative Shutdown Capability

a. Inspection Scope

The inspectors reviewed the licensee's systems required to achieve alternative safe shutdown to determine if the licensee had properly identified the components and systems necessary to achieve and maintain safe shutdown conditions. The inspectors also focused on the adequacy of the systems to perform reactor pressure control, reactivity control, reactor coolant makeup, decay heat removal, process monitoring, and support system functions.

The team conducted selected area walkdowns to determine if operators could reasonably be expected to perform the alternate safe shutdown procedure actions and that equipment labeling was consistent with the alternate safe shutdown procedure.

The review also looked at operator training as well as consistency between the operations shutdown procedures and any associated administrative controls.

b. Findings

No findings of significance were identified.

.7 Circuit Analyses

a. Inspection Scope

The inspectors reviewed the licensee's post-fire safe shutdown analysis to verify that the licensee had identified both required and associated circuits that may impact safe shutdown. On a sample basis, the inspectors verified that the cables of equipment required achieving and maintaining hot shutdown conditions, in the event of fire in the selected fire zones, had been properly identified. In addition, the inspectors verified that these cables had either been adequately protected from the potentially adverse effects of fire damage, mitigated with approved manual operator actions, or analyzed to show that fire-induced faults (e.g., hot shorts, open circuits, and shorts to ground) would not prevent safe shutdown. In order to accomplish this, the inspectors reviewed electrical schematics and cable routing data for power and control cables associated with each of the selected components. In addition, on a sample basis, the adequacy of circuit protective coordination for the safe shutdown systems' electrical power and instrumentation busses were evaluated

b. Findings

No findings of significance were identified.

.8 Communications

a. Inspection Scope

The inspectors reviewed, on a sample bases, the adequacy of the communication system to support plant personnel in the performance of alternative safe shutdown functions and fire brigade duties. The inspectors verified that plant telephones, page systems, sound powered phones, and radios were available for use and maintained in working order. The inspectors reviewed the electrical power supplies and cable routing for these systems to verify that either the telephones or the radios would remain functional following a fire.

b. Findings

No findings of significance were identified.

.9 Emergency Lighting

a. Inspection Scope

The inspectors performed a plant walkdown of selected areas in which a sample of operator actions would be performed in the performance of alternative safe shutdown functions. As part of the walkdowns, the inspectors focused on the existence of sufficient emergency lighting for access and egress to areas and for performing necessary equipment operations. The locations and positioning of the emergency lights were observed during the walkdown and during review of manual actions implemented for the selected fire areas and/or fire zones.

b. Findings

No findings of significance were identified.

.10 Cold Shutdown Repairs

a. Inspection Scope

The inspectors reviewed the licensee's procedures to determine whether repairs were required to achieve cold shutdown and to verify that dedicated repair procedures, equipment, and material to accomplish those repairs were available onsite. The inspectors also evaluated whether cold shutdown could be achieved within the required time using the licensee's procedures and repair methods. The inspectors also verified that equipment necessary to perform cold shutdown repairs was available onsite and properly staged.

b. Findings

No findings of significance were identified. The inspectors noted that no repairs were required at Clinton Power Station to achieve cold shutdown.

.11 Compensatory Measures

a. Inspection Scope

The inspectors conducted a review to verify that compensatory measures were in place for out-of-service, degraded or inoperable fire protection and post-fire safe shutdown equipment, systems, or features (e.g., detection and suppression systems, and equipment, passive fire barriers, pumps, valves or electrical devices providing safe shutdown functions or capabilities). The inspectors also conducted a review on the adequacy of short term compensatory measures to compensate for a degraded function or feature until appropriate corrective actions were taken.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems (71152)

.1 Routine Review of Condition Reports

a. Inspection Scope

The inspectors reviewed the licensee's corrective action program procedures and samples of corrective action documents to verify that the licensee was identifying issues related to the fire protection program at an appropriate threshold and entering them in the corrective action program. The inspectors reviewed selected samples of condition reports, work orders, design packages, and fire protection system non-conformance documents.

b. Findings

No findings of significance were identified.

4OA6 Management Meetings

.1 Exit Meeting Summary

On July 25, 2008, at the conclusion of the inspection, the inspectors presented the inspection results to Mr. M. E. Kanavos and other members of licensee management. The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

.2 Interim Exit Meetings

No interim exits meetings were conducted.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee

T. Parrent, Fire Protection Engineer
S. Deal, Fire Marshal
C. Pragman, Fire Protection Engineer (Corporate)
A. Darelius, Nuclear Oversight
M. Taylor, Fire Protection Engineer (Corporate)
R. Frantz, Regulatory Assurance
M. Kanavos, Plant Manager
R. Weber, Site Engineering Director
S. Gackstetter, Training Director
J. Rappeport, Chemistry Manager
S. Clary, Engineering Programs Manager
R. Chickering, Engineering
G. Mosley, Engineering
M. Evans, Work management
R. Philips, Maintenance Work Planning
J. Peterson, Operations

Nuclear Regulatory Commission

C. Kemker, Senior Resident Inspector
D. Lords, Resident Inspector
J. Lara, Chief, Engineering Branch 3, DRS

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened, Closed, and Discussed

None

LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety, but rather, that selected sections of portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

CALCULATIONS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
19-AN-04	480V ESF Swgr Bkr and Assoc Upstream Relay Settings	12
19-AN-08	4160V ESF Swgr Buses 1A1 and 1B1 Mtr Relay Settings	3
CC-AA-309-1001	Fire Loads in Clinton Power Station Fire Zones	3
IP-C-0006	Diesel Fuel Oil Stor Tanks DIV I, II, and III Tank Vol Cal	0
IP-C-0111	DG Day Tank Level and Volume Cal, Day Tank and Fuel Storage Tank Level Setpoint and Required Indications	2
IP-M-0532	Safe Shutdown Analysis for Various Fire Areas	0
IP-M-0177	Fire Loads In Clinton Station FZ R-1i, R-1r and R-1s	December 27, 2007

CORRECTIVE ACTION PROGRAM DOCUMENTS ISSUED DURING INSPECTION

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
AR00788577	NRC ID: Smell of Fuel Oil in DIV 3 FOST Room	June 20, 2008
AR00797505	USAR App E Dwg FP-12B Does Not Show AWS Sys	July 17, 2008
AR00799195	Reference #6 in Calculation IP-C-0006 Is Unclear	July 22, 2008
IR00800021	USAR Appendix F Table and CPS 4003.01 Typo Errors	July 24, 2008

CORRECTIVE ACTION PROGRAM DOCUMENTS ISSUED PRIOR TO INSPECTION

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
AR00345135	FP 225 Tubing Pinhole (High Pressure for Flow Gage)	June 17, 2005
AR00458628	Fire Protection Leaking Underground by Service Bldg	February 25, 2006
AR00518473	1LL 618P10E Emerg Light Will Not Stop Fast Charging	August 9, 2006
AR00519285	1LL 618P10E SSD Pathway Light Has Failed	August 12, 2006
AR00657922	54BP02E Remained in Fast Charge Past 24 Hours	August 7, 2007
AR00661700	CPS IN 2007-026 Actions – Comb Epoxy Flr Coating	October 31, 2008
AR00665956	Fire Load Issues in Approved Vital Area Storage Areas	August 29, 2007
AR00693786	Inadeq Extent of Cond Review of SSD Light Restraint	November 2, 2008
AR00717441	IEMA Inspector Questions on Transient Combustible	January 2, 2008

CORRECTIVE ACTION PROGRAM DOCUMENTS ISSUED PRIOR TO INSPECTION

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
	Material	
AR00721120	Potential Green NCV for FP Program Violations	January 11, 2008
AR00730611	New Emergency Light Batteries Are Not Like For Like	February 2, 2008
AR00767151	FP FASA (AR699105) SSD Lighting Testing Issues	April 24, 2008
AR00782513	NOS ID Transient Comb Under Cable Tray 781FB	June 2, 2008

DRAWINGS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
1 K 2842	Contract 73011 Drawing DCP 1 Sheet 1 General Plan 14' 0" I.D. x 44' 0" T.L. Horiz. Tank	6
2 K 2842	Contract 73011 Drawing DCP 2 Sheet 1	2
A26-1002-01A	Control Building HVAC Floor Plan Area 2	T
A26-1002-03A	Control Building Mezzanine Floor Plan Area 2	G
A26-1003-02A	Auxiliary Building Switchgear Floor Plan Area 2	E
A29-1000-01A	Diesel Generator Building Floor Plan Area 2	M
E02-1DG99, Sheet 9	EDG 1A Control Schematics	J
E02-1DO011	Loop Schematic Diagram: Diesel Fuel Oil System (DO) DG Fuel Oil Storage Tank 1DO01TA Level	B
E02-1FW99, Sheet 15	Reactor Feed Pump 1C Schematic Diagram	W
E02-1RH99, Sheet 507	RHR1A Cont Spray Vlv 1E12-F028A and Pump 1A LPCI Inject Spray Vlv 1E12-F042A Schematics	L
E02-1RH99, Sheet 529	RHR Pump 1A Schematic Diagram	F
E02-1RS99, Sheet 107	Remote Shutdown System Schematic Diagram	G
M05-1039	P&ID Fire Protection (FP) Turbine Building Sheet 008	AL
M05-1056	P&ID Plant Service Water (WS) Sheet 002	AH

MODIFICATIONS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
EC0000336376	Eval of Electrical Temporary Modification Cable Storage in Laydown Areas in The Plant	April 23, 2002

PREFIRE PLANS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
CPS1893.04M103	707 Auxiliary: RCIC Pump Room Prefire Plan	4
CPS1893.04M200	723' – 778' Drywell Prefire Plan	4
CPS1893.04M220	755' Containment Prefire Plan	4
CPS1893.04M231	778' Containment – West Prefire Plan	4
CPS1893.04M250	828' Containment Prefire Plan	5
CPS1893.04M310	719 Control: HVAC Equipment Area Prefire Plan	5a
CPS1893.04M400	712' Fuel: Basement (F-1A, B C, D, E, F, G, H, I, P)	4
CPS1893.04M600	702 Radwaste (South): Basement Prefire Plan	3
CPS1893.04M611	702-725 Radwaste (North): Interm Flr Prefire Plan	3
CPS1893.04M620	737 Radwaste: Shops and Storeroom Prefire Plan	7a
CPS1893.04M625	737 Radwaste: Paint and Oil Storage Rm Prefire Plan	3a
CPS1893.04M700	712 Turbine: Basement Prefire Plan	5b
CPS1893.04M703	712 Turbine: Condenser Prefire Plan	5a
CPS1893.04M720	762 Turbine: Turbine Auxiliaries Prefire Plan	5
CPS1893.04M810	722 Service Building Basement Prefire Plan	1
CPS1893.04M811	737 Service Building Ground Level Prefire Plan	1a

PROCEDURES

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
40030104LSA01	CPS Job Performance Measure - RCIC Startup at the RSD Panel with Flow Controller Failure	0
CPS1893.04	Fire Fighting	11c & 11d
CPS1893.04M132	Fire Zone A-2n, DIV 1 Switchgear Pre-fire Plan	4
CPS1893.04M132	Fire Zone CB-1f, General Area	4
CPS1893.04M132	Fire Zone CB-1i, Control Room HVAC	5
CPS1893.04M132	Fire Zone CB-4, Cable Spreading Room	9
CPS3001.01	Preparation for Startup and Approach to Critical	24e
CPS3213.01	Fire Detection And Protection	25d
CPS3822.17C001	Safe Shutdown Emergency Light Checklist	11a
CPS4003.01	Remote Shutdown (RS)	14a
CPS9277.11	Fire Hose Hydrostatic Test	25a
CPS9277.22	Yard Fire Hose Inspection Test	25

PROCEDURES

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
CPS9437.05	RSD System RCIC Flow E51-N003 Channel	39b
ER-AA-5400-1002	Buried Piping Examination Guide	1
MA-AA-716-010	Maintenance Planning Procedure	11
MA-MW-716-010-1000	Passport Work Planning Manual	10
ME-06.00	Guidelines for Determining Fire Loads and Preparing Fire Load Calculations	8
OP-AA-201-002	Attachment 1 Fire Event Report	3
OP-AA-201-003	Attachment 1 Fire Drill Report	6
OP-AA-201-009	Control of Combustible Material	7
OP-CL-2001-009-1001	Control of Transient Combustible Material	0
OP-MW-201-007	Fire Protection System Impairment Control	6

REFERENCES

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
-----	FP Program Binder NFPA Conform Eval GL86-10	September 16, 2005
-----	List of Emergency Lighting Units	July 2008
-----	Fire Drill Procedure with Local Fire Department	August 10, 2005
-----	Fire Drill Procedure with Local Fire Department	December 6, 2006
FASA-08	Focused Area Self-Assessment of Fire Protection	May 16, 2008
GL86-10-2001-10-01	GL86-10 Eval for Elimination of 1E51-F095 Valve	November 7, 2001
GL86-10-2001-10-02	GL86-10 Eval Impact EC332051 on Cal IPM-0177	October 30, 2001
GL86-10-2001-10-03	GL86-10 Eval Impact EC332120 on Cal IPM-0177	October 31, 2001
GL86-10-2005-09-01	GL86-10 Eval for NFPA Code Conformance Eval	September 16, 2005
JPM218	CPS Job Performance Measure - Place RHR A In Shutdown Cooling at the RSD Panel	00
NFPA No. 13	Sprinkler Systems	1976
NFPA No. 72	Automatic Fire Detectors	1974
NFPA No. 80	Fire Doors and Windows	1975
NO-AA-10	Quality Assurance Topical Report	81
NOSA-CPS-05-10	Fire Protection NOS Audit	June 15, 2005
NOSA-CPS-07-09	Fire Protection NOS Audit	December 19, 2007
OP-AA-201-005	Fire Brigade Qualification	6
OP-CL-101-102-1001	CPS Minimum On-Shift Staffing Functions	2
TCP 2007-08-002	Transient Combustible Permit (FR#665956)	August 28, 2007
TQ-AA-127	Fire Brigade Training Program	4

VENDOR DOCUMENTS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
-----	Teledyne Big Beam Six Volt Emergency Lighting	May 1979

WORK DOCUMENTS

<u>Number</u>	<u>Description or Title</u>	<u>Date or Revision</u>
WO00763404 01	0337.81B21 CF Smoke Det (Ion Smoke Detector)	December 16, 2006
WO00791825 01	9601.01J20 VI Interam (3M) Cable Tray Wrap Insp	December 26, 2006
WO00895716 01	9601.01C002 Damper Inspection of VF Dampers	March 3, 2008
WO00898367 02	9601.01C20 VI Inaccessible Fire Rated Sealg Dev	January 21, 2008
WO00931157 01	9476.03I22 OP Halon Sto Tk Weight & Pres Verif	February 29, 2008
WO00948620 01	Perform 3822.17 C001 Battery Discharge Test	July 30, 2007
WO00948937 01	9071.02S20 OP Fire Pump B Capacity/Controls Test	August 9, 2007
WO00972152 01	9071.02S20 OP Fire Pump A Capacity/Controls Test	November 17, 2007
WO00982208 01	Perform 3822.01 C008 For All EINs in Diesel	September 2, 2007
WO01085232 01	Change-out Flr Drain Socks in Power Block 1870.02	February 21, 2008
WO01094790 01	9601.06R20 VI Fire Door Hardware Inspections	June 9, 2008
WO01127901 01	RS Emerg Lighting Verification per MA-AA-723-350	May 10, 2008
WO01144935 01	9377.01A20 OP FP Battery Pilot Cell CK	July 8, 2008

LIST OF ACRONYMS USED

ADAMS	Agency-wide Document Access and Management System
DRP	Division of Reactor Projects
DRS	Division of Reactor Safety
EDG	Emergency Diesel Generator
FP	Fire Protection
IMC	Inspection Manual Chapter
IP	Inspection Procedure
IPEEE	Individual Plant Examination for External Events
NRC	U.S. Nuclear Regulatory Commission
NRR	Office of Nuclear Reactor Regulation
OA	Other Activities
PARS	Publicly Available Records System
SDP	Significance Determination Process
WS	Service Water